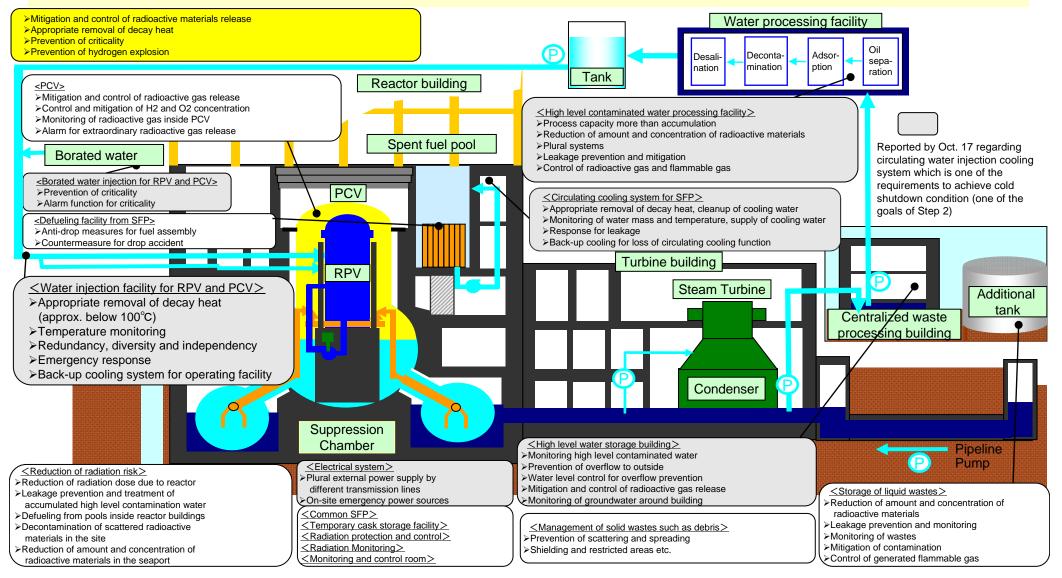
3. Mid-and-long Term Roadmap



Concept for Mid-term Safety Security (Basic Target Outline)

- NISA released "Concept for Mid-term Safety Security" on Oct. 3, 2011, which covered the safety security concept until the start of decommissioning work.
- We reported to NISA on the operating plan as well as the safety assessment results regarding the circulating water cooling system. Other systems etc. shall be reported on as well in a timely manner.



Story Behind the Mid-and-long-term Roadmap, Safety Securement

Per the order issued on November 9, 2011 by Mr. Edano, the Minister of Economy, Trade and Industry and Mr. Hosono, the Minister for the Restoration from and Prevention of Nuclear Accident, this roadmap was drafted by TEPCO, ANRE and NISA and finalized at the government and TEPCO's mid-and-long-term countermeasure conference on December 21, 2011.

<Basic Policy towards Addressing the Mid-and-long Term Issues>

- [Policy 1] Systematically tackle the issues while placing the top priority on the safety of local citizens and workers.
- [Policy 2] Move forward while maintaining transparent communications with local and national citizens to gain their understanding and respect.
- [Policy 3] Continuously update this roadmap based on the on-site situation and the latest R&D results etc.

[Policy 4] Harmonize the respective efforts of TEPCO, ANRE, and NISA to achieve our goal.

<The Overall Plan to Secure Mid-and-long-term Safety>

- In the upcoming three years, TEPCO will implement the operation and management plan for their facilities based on "SAFETY DIRECTIVE "Ensuring Mid-term Safety"" issued by NISA. NISA will review and assess TEPCO's reports based on their investigative standards and thus will secure safety.

- Mid-and-long-term actions will be implemented as well. TEPCO will conduct safety and environmental impact assessment at each juncture when TEPCO consider concrete work procedures for each task. NISA will assess and confirm these procedures prior to implementation, thus ensuring safety.

Mid-and-long Term Roadmap

<Primary Target>

Present all possible schedules pertaining to the main on-site works and R&D.

- <Target Timeline and Holding Points>
- Established all possible target timelines in the upcoming 3 years, which are updated and released on a yearly basis.
- Regarding the schedules after 3 years, established holding points, which are significant to judge whether to go ahead in accordance with the schedule, to implement additional R&D, or to re-schedule the process.

| Present (Com | pletion of Step 2) With | nin 2 Years Within 1 | IO Years After 30-40 Yea |
|--|---|---|---|
| Step 1, 2 | Phase 1 | Phase 2 | Phase 3 |
| <achieved conditions="" stable=""> -Condition equivalent to cold shutdown -Significant Suppression of Emissions</achieved> | Period to the start of fuel removal from the spent fuel pool (Within 2 years) | Period to the start of fuel debris removal (Within 10 years) | Period to the end of decommissioning (After 30-40 years) |
| | -Commence the removal of fuels from the spent fuel pools (Unit 4 in 2 years) | -Complete the fuel removal from the spent fuel pools at all Units | -Complete the fuel debris removal (in 20-25 years) |
| | -Reduce the radiation impact due to additional emissions from the whole site and radioactive waste generated after the accident (secondary waste materials via water processing and debris etc.) Thus maintain the effective radiation dose to be less than 1 mSv / year at the site boundaries caused by the aforementioned. | -Complete preparations for the removal of fuel debris such as decontamination of the insides of buildings, restoring PCVs and filling PCVs with water. Then commence the removal of fuel debris (Target: within 10 years) | -Complete the decommission (in 30-40 years) -Implement radioactive waste processing and disposal |
| | -Maintain stable reactor cooling and accumulated water processing and improve their credibility. -Commence R&D and decontamination towards the removal of fuel debris | -Continue stable reactor cooling -Complete the processing of accumulated water | |
| | -Commence R&D of radioactive waste processing and disposal | -Continue R&D on radioactive waste processing and disposal, and commence R&D on the reactor facilities decommission | |
| Actions towards implemented. | s systematic staff training and allocation, m | otivation improvement, and securing of worker | 's' safety will be continuously |

Main Schedule of Mid-and-long Term Roadmap (1/2)

| | | | | Phase 1 | | Phase 2 | | | | | | | | Phase 3 | | |
|--|-----------------------------|-----------------------|--|---|--|--|--|------------------------------------|---|-------------------|-------------------|-------------------------|----------------------------|---|--|--|
| | | | Period to the sta | art of fuel removal from t | | | | Period to the | Period to the end of decommissioning | | | | | | | |
| | | 1 | FY2012 | FY201: | | | FY2016 F | Y2017 FY2018 | FY2019 | FY2020 | FY2021 | FY2022~ | | | | |
| | | | | | | | Early) | | (Mid) | | (Late) | Within 1 | | After 20-25 years | After 30-40 years | |
| Prin | nary Tara | gets Step 2 C | ompleted V | | | Removal Start Init 4) V | | | | | | Fuel Debris R (First | emoval Start Unit) 7 | Fuel Debris Removal Completed (All Units) V | Decommissioning Completed (All Units) | |
| | | | Ongoin | g Monitoring of Reac | tor Cold Shutdov | vn States (Maintain v | vater injection and mo | titoring using ter | nperature and pressure par | rameters etc.) | | | | | : On-site Work | |
| Plan for Reactor Partial Internal PCV Inspection VTarget: Complete Switch to Water Withdrawal from Reactor Building (or VTarget: Start of Circulating Water Cooling within | | | | | | | | part of PCV) | | | : R&D | | | | | |
| | Cooling | | | · · · · | | | | | ter Cooling (water withdraw | | within the | Circulati | ng Water Cooling | via Water | : Considerations Conditions for Next | |
| Plan for | | | Improving the I | Reliability of Circulati | ing Water Cooling | (water withdrawal fr | | from reactor b | uilding (or lower part of PC | CV)) | | Withdraw | val from PCV (sho | ort loop) | Information Flow | |
| Maintaining | | | | Facibility Study of the Early Realization of Circulating Loop eithin the Building | | | | HP3-1]; PCV | (*2) From (*4) | Reactor Buildi | | | tc. (Considerat | ion based on | \checkmark | |
| Plant in an Ongoing | | | Processing Accumu | lated Water via Existing | V larget:Improv | e Reliability of Existin | building V | later Leakage | | 1 | | | lete Accumulat | ed Water Processing in | | |
| Stable State | | | Processing Facilities | | | | ng via Reliability Impro | ved Water | HP 1-1 Completion of Stoppin Leakage between Read | tor and Turbine B | ater Buildings | Turbine/Read | tor Building | | | |
| | Plan for Accumu | lated Water | Improving the Reliabilit | ty of Existing Facilities etc. | Processii Implement work accordi | ng Facilities | | Consideration of Circula | and Repairing Lower P | art of the PCV | | | | | | |
| | Process | | Line Decrease Purification/Restoring | g Sub-drain | the consideration result Step-by-step Op | eration of Sub-drain -> Decrea | sing Ground Water Inflow (Decre | Line Decrease asing Accumulated | / (if necessary) / | > | | Descention | | d and Damatania stad Wet | | |
| | | | Pit | Bypassing Ground | Water) Dec | reasing Ground Water Inflow (D | ecreasing Accumulated Water) | • | _ <u>_</u> | • | | Processi | ng of Undergroun | id and Decontaminated Wate | er etc. | |
| | | | Installation of Multi-nu Removal Facilities | clide Purification o | of the Accumulated Water | | <u>kk</u> | | Reduction of Accumulated Wa | | · |) | | | | |
| | | | | Water Shielding Wal | lls Installation | Vlarg | et: Reduction of the I | disk of Expanded | Sea Water Contamination | when Contamir | nated Water I | _eaks | | | | |
| | Plane to | Mitigate | Additional Silt Fence Installation | ⊽Target : | | dioactive Substance | Concentrations in Se | a | | | | | | | | |
| | Sea Wat | er | Covering Seabed Soil of the Intake Canal | | | announced density) | | | | | | | | | | |
| | Contam | ination | Girculating Sea | awater Purification (o | | away/Anchor Ground | | | | | | | | | | |
| | | | | o o to this i | | | | ater and Seawate | er Monitoring (ongoing) | 1 | | | | | | |
| Plan to Reduce | | | | | VTarget: Attain | ment of Dosage Belg | - | - | to Sources such as New | Emissions of Re | adioactive Su | hstances etc | from the Pow | er Station as a Whole | | |
| Radiactive | | Rubble | educe Radiation Dose fro | m Stored Rubble etc. via Shielding | 2 | | | | | | | | | | | |
| Dosage in the Power | Radioac | | Development of Mi | d-and-long term Storage Managen | Continue Re | duction Efforts | | Continue | Storage and Reinforce | | | | | | | |
| Station as a | ve Wast Manager | m Secondary | | | | | | | | | | | | | | |
| Whole, and to Mitigate | ent and Dose Reductio | | | essing via Shielding etc. ose from Secondary Waste | Continue Re | duction Efforts | | Continue Store | age | | | / | Facility Facility Facility | Replacements Carry out needed) Disposal S | to | |
| Sea Water Contaminatio | | ctio Water | Assess Characte | eristics of Secondary Wa | aste from Water | Facility Replacemen | t | | | 1 | | | | (*5): Go to " | Radioactive | |
| _ | n at the | Processin | | Storage Container Lifes ng/disposal will be co | | Plan Development adioactive waste prod | _/ cessing∕disposal plan) | | | | | | | Waste Proces Plan [‴] | ss/Disposal | |
| | Site Boundari es | ri Gaseous∕ Liquid | PCV Gas Control S | System Installation active material emissions from F | PCV) | | | | | | | | | | | |
| | | Waste | Land/Sea | Area Monitoring (ong | oing) | | - | | | | | | | | | |
| | Plan for | Field Test | | | VTarget: Reduc | e Radiation Dose at | | Implement per p | artner companies needs) | | | | | | | |
| | | | 1 | | - | + | 1 | a in conjunction | with efforts to reduce radi | ation dose outsi | de of the site | :) | | | | |
| | | | 1 | | 1 | y via maintenance an Survey of Rubble etc.X | - | | | | | | | | | |
| | | | 1 | of Fuel Removal Measures/ In | 1 | Planning | | | oble, facety installation etc.) Fuel R | | | | | | | |
| | SFPs of | Units 1–4 | [Unit 2] Consideratio | n/Preparation of Decontaminat | | | Shielding, Facility Survey Planni | 1 | n, Repairs Fuel Remov | | | | | | | |
| | | | [Unit 3] Removal of Ru | (*3) bble (upper part of reactor build | Target: Complete I | acilities Removal of Rubble | <mark>⊽Start Fuel Removal (Targ</mark> Fuel Rom | | | | | | | | | |
| Plan for Fuel | | | | ▼Target: Complete | e Debris Removal 🛛 🔊 | VStart Fuel Removal (Tar | ву | | | | | | | | | |
| Removal from Spent | | | | ble (Upper) Install Fuel Handlin, buildings) and Cover for Fue | g Facilities Removal I Removal the pools/ | No Debris in Fuel r Fuel Suevey Fuel r | removal | | | L | | | | | | |
| Fuel Pool | | ĺ | Site Harbor Restoration (Restoration of Crane | on and Road) Shallow D | ration of Praft Quay | Storage of Fuel | Assemblies Removed | from SFP (store | e/manage) | | | | | Carry out | | |
| | Common Pool | | Cask Manufacturing (s | step-by-step) Cask M | anufacturing/ Delivery (s | step-by-step) | | | | | 1 | | | | | |
| | 20.1110 | | Common Pool | Restoration Common Pe | ool Fuel Removal and Fac | il <mark>ty Modifications</mark> | | | | | | mination of M | | | | |
| | | | | | | | Common Pool Modific | ations for Fuel S | Storage | L | Repro | cessing and \$ | Storing Spent F | uel | | |
| 1 | R&D | | Long-term I | Integrity Assessment | | ies Removed from SF | | | | | | | | | | |
| | | | | | Consider | Handling Method of I | Damaged Fuels from t | he SFPs | | | | | | | | |

Main Schedule of Mid-and-long Term Roadmap (2/2)

| | | Phase 1 | | | | | | Phase 2 | | | | | | | Phase 3 | | |
|--|---|---|-----------------------|---|--|--------------------------------------|--|---|---|--|--|---|--|---|---|---|--|
| | | Period to the s | tart of fu | | he spent fuel pools | | Period to the start of fuel debris removal | | | | | | | | I to the end of decommissioning | g | |
| | | | | FY2012 | FY2013 | FY201 | 4 FY201: (Early) | 5 FY2016 | FY2017 | FY2018 | FY2019 | FY2020 (Late) | FY2021 | FY2022~ | After 20-25 years | 40.0040 | |
| | 1 | | | (*3) | within | z years | | | (MI) | A/ | | (Late) | WITHIN | U years | Alter 20-20 years | After 30-40 year | |
| | Decontamination of the Inside of the | Decontamination Decontaminating and field test) | Technolo Equipment | pgy Investigation / F nt Development (incl | Remote luding field survey | ⊽Target: Esta | blish Decontamiı | nating Robot Tech | nology | | | | | | | : On-site Work : R&D :Considerations | |
| | Reactor Building | | | Internal Build | ing Decontaminatio | h and Shielding et | :c. > | (Ongoi | ng) | \rightarrow | | | | | | onditions for Next Tasks | |
| | | | | | | | ⊽Target: Se | cure Access Rout | e via Decontamir | ation | | | | | · Inf | formation Flow | |
| | Inspections of Leakages inside PCVs | R&D for PC stopping inte | / Leaka ar-buildi | ige Point Surve ing water leaka | y/ Repairs (inclu ge) | ding | | HP 3-1 Deten PCV, Metho | mination of Method Determination Wat ds | s for Repairing er Stop HP 3-3 | Determination o | of Methods to F | ★ Repair Uppe | r Parts of PCV | | | |
| | Inter-building Leak Stop PCV Repair | Design, Mar Survey Equ | | re and Test, etc | . of PCV Leakage | Point | | | | | | | ★ :Determine Priority of Based on Plant Status at | | | | |
| | Filling Up the PCV/RPV | | | | Ir | | akage Points (inclu f R&D results) | Inter-building | V's Lower Part/ Stoppin Water Leakage/ Filling th r Part with Water | Repair | r the PCV's Upper Part/I the Upper Part with Water | Filling | | | | | |
| el Debris | Inspections and Samplings of the insides of the PCVs | | address | | such as the intern | al inspection | | | | Completion of Parts of PCV PCV Internal Methods | f Flooding of Lower , Determination of Investigation | | letion of Floodi mination of RP ods | ng of Upper Parts o / Internal Investiga | of PCV, tion | | |
| moval n | Fuel Debris Removal Technology Preparation | Design, Mar | nufactu | re and Test, etc | . of Internal PCV | | \ | | Design, Manufactu Internal RPV Insp | | oriand E | lopment of Techr Equipment for Debris Removal | 3 | | on of Fuel Debris Removal Me of Preparation of Fuel Debris | | |
| | Fuel Debris Removal Work | | | | Ir | | e of PCV (including R&D result) | field | | Internal PCV Inspecti and Sampling | on | Internal RPV Inspection an Sampling | nd | Fuel Debris | Removal | | |
| | Safe Storage, Processing and Disposal of Removed Fuel Debris | R&D for Fue Processing | l Debris | | Development of Investigation of Exis Technology for Plac | ting Technology, Se | | ystem as well as Develo Mock-up Test | oment of Safety Asse | ssment Technology, | Development of | | | Storage | earry out, | Processing and Disposa | |
| | | - | 1 | - | rocessing/ Disipo | sal Technology | Test and Eva | Iluation Using Mocl | k−up Debris Samı | le \ Tes | t and Evaluation | Using Actual D | ebris Sampl | | >> Processi | nation of ing∕Disposal ₃ of Fuel Debris | |
| | Reactor Building Container | Establish I | Measuri | ng Method to W | leigh Fuel Debris | | | Eullpagel | + Consideration/I | Reaster | Building Container Ins | etallation 44 | | | | | |
| | Installations etc. | | | _ | | | | Full-scale | | Jesign Reactor | | | | | | | |
| | RPV/PCV Integrity Maintenance | | | | sment Technolog ucing oxygen diss | | | nitrogen bubbling) | | Repairs / C | orrosion Prevention | n Measures (imple | ement additiona | corrosion preventi | on measures as needed) | | |
| | Reactor Dismantling Plan | Investigate a Database Est | | | | asic Database (tor Facility Disr | contamination st mantlement | tatus | | R | D of Remote Dis | mantlement etc | | P Determinati Methods. D Design/Manu Machines/Fac | vilities Weste Com | amination Engineering for Demolition Waste. Disposal of Demolitio apletion of R&D. | |
| actor smantling | | | | | | | | - Lille of Descent I | | Cons | ideration of an In | stitutional Fran | nework | Systema Physic | al Form of | ntlement | |
| & Radioactive Waste Processing/ Disposal Plan | Radioactive Waste Processing/Disposal Plan | R&D for Safe | ty Conf | firmation of the | Processing/Disp | osal of Radioac | Co | ability of Present I oncept according t Charact | Disposal bo Waste ceristics | | Confirm Saf | fety Plan for Wa | aste HP | HP 5-3 | HP 5-4 Installatio Manfactu the Wast | e Form | |
| | | Development | | | Assessment of | | <u> </u> | tablishment of Disp | · · · | \geq | | | | Form Ma Equipme | nufacturing of W nt | lisposal | |
| ganization | & Staff Planning | Processing/D | - | / | 1 | al Waste Dispos ner companies) | | R&D of Er | | Assessment for | Processing/Disp | oosal | | Continue R&D to | o Improve Safety of Processin | ng/Disposal | |
| - | | | + | | ł | - | I | 1 | | | 1 | | | E | | | |
| rker Safet | y Plan | | Cont | tinue to Promot | e Safety, Maintai | n and Improve F | adiation Protect | tion Measures, Con | tinuously Maintai | n Medical Care | System | | | | | | |

Organizational Structure of the Mid-and-long Term Roadmap

Established "Steering Committee" and "R&D Promotion Headquarters" under Government and TEPCO Mid-and-long Term Countermeasures Committee on December 21 2012 which is held every month to monitor the progress to ensure steady implementation of the midand-long term roadmap.

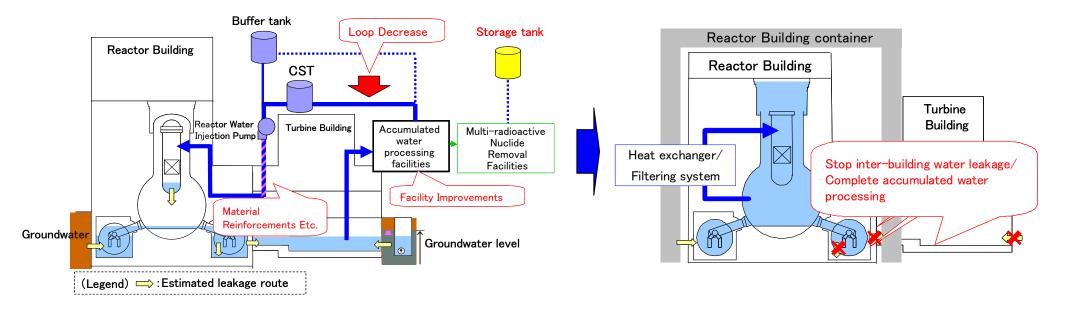
➢As we are facing many difficulties in research and development that are unprecedented and challenging even from a world-wide perspective, we will work hand-in-hand with our domestic and overseas partners, and compile wisdom and knowledge from all over the world as we move forward.

Concerning the onsite work, TEPCO will maintain the current structure with approximately 400 partner companies and established "Fukushima Daiichi Countermeasures Project Team" in February 2012 at Headquarters as a specialized organization to deal with mid-andlong term roadmap issues. Improvement of the work environment and systematic staff training will contribute to securing the performable organization and staff.

>Further enhancement of R&D promotion framework including preparation of R&D facilities will continue to establish the best framework to deal with challenges becoming clearer so far.

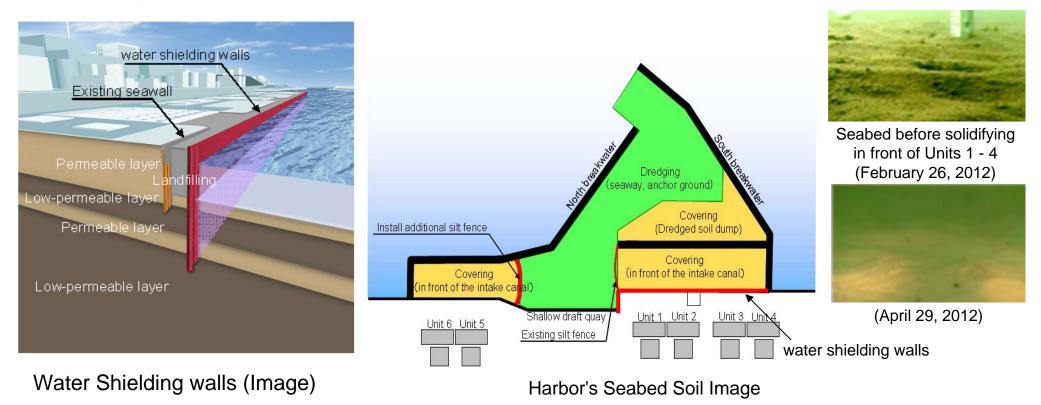
Target Timeline: 1) Reactor Cooling, Accumulated Water Processing

- By examining the reliability of the system, system improvements will be continuously implemented such as polyethylene pressure-proof hoses for injection and circulation lines.
- Since the excess water volume is increasing due to groundwater inflow to the buildings, we will take countermeasures to reduce the excess water: pumping up groundwater to bypass its outflow route to the sea, installing Advanced Liquid Processing System (ALPS) and so on. In addition, we will develop a tank operation plan to store the processed water without overflow.
- During Phase 2, after stopping the water leakage between the turbine and reactor buildings and completing the repair work of the lower parts of the PCVs, we will finish the processing of water accumulated inside the building. In order to achieve more stable cooling, scaling down of the circulation loop is being considered.



Target Timeline: 2) Plans to Mitigate Sea Water Contamination

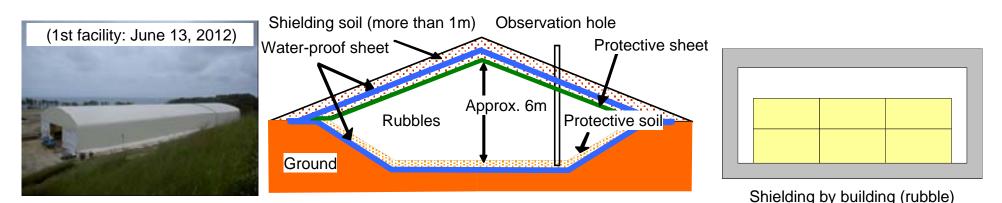
- In case underground water be contaminated, water shielding walls will be built by mid FY2014 in order to prevent underground water from flowing into the ocean.
- Covering and solidifying seabed soil in front of the intake canal will prevent the diffusion of radioactive materials in the soil (completed in July 2012). By the end of the first half of FY2012, we will reduce radioactive materials in the seawater inside the site port to the level below the limit for the outside of environment surveillance areas as determined by a notification of the government.
- Afterwards, while maintaining the installed facilities, underground water and sea water etc. will be continuously monitored.



Target Timeline: 3) Radioactive Waste Management and Dose Reduction at the Site Boundaries, 4) Onsite Decontamination Plan

- By the end of FY 2012, we will reduce the effective radiation dose at the site boundaries, stemming from additional emissions from the whole site and radioactive waste stored on the site after the accident (secondary waste materials via water processing and rubble etc.), to below 1 mSv / year. Target of radiation dose reduction is set for each released radioactive material and stored radioactive waste, and its reduction effect and additional countermeasure are considered every quarter.
- By around the end of 2012, we will develop a mid-to-long term plan to manage and secure storage areas which is estimated from the past records and future work plan. The plan also will cover how we minimize the radiation effect at the site boundaries: shifting the temporary facility to the one that withstands long term usage.
- In order to reduce the radiation exposure of the public and site workers and to improve work environment, systematically and in a step-by-step manner, we will implement decontamination work in accordance with the area classification: administration area, work area and access area.

In May 2012, office area in the seismic isolation building became non-controlled area. Within 2012, commuting bus stop and work area where main gate security guards are stationed will be decontaminated and taken shielding measures, etc. From 2nd phase, we will move forward with the decontamination work inside the site in coordination with the outside dosage reduction.



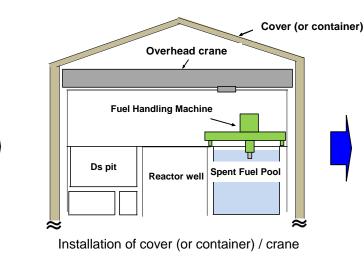
Preparation work for temporary storage facilities shielded by soil (completed)

Target Timeline: 5) Plans to Remove Fuels from Spent Fuel Pools

- >To start fuel removal from Unit 4 within 2 years after completing Step 2 (within 2013).
- To start fuel removal from Unit 3 approximately 3 years after completing Step 2 (end of 2014).
- To develop a fuel removal plan from Unit 1 based on the experiences in Units 3 & 4 and investigations of the rubble situation, and will complete fuel removal during the Phase 2.
- To develop a fuel removal plan from Unit 2 in light of the investigation results of the installed facilities to be conducted in consideration of decontamination results inside the buildings and will complete fuel removal during Phase 2.
- >To complete fuel removal from all Units during Phase 2.
- >To determine reprocessing and storing methods for removed fuels during Phase 2.



Debris removal from the upper part of reactor building



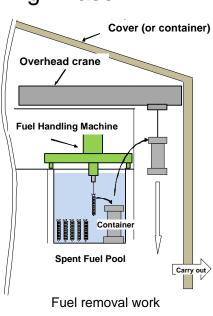
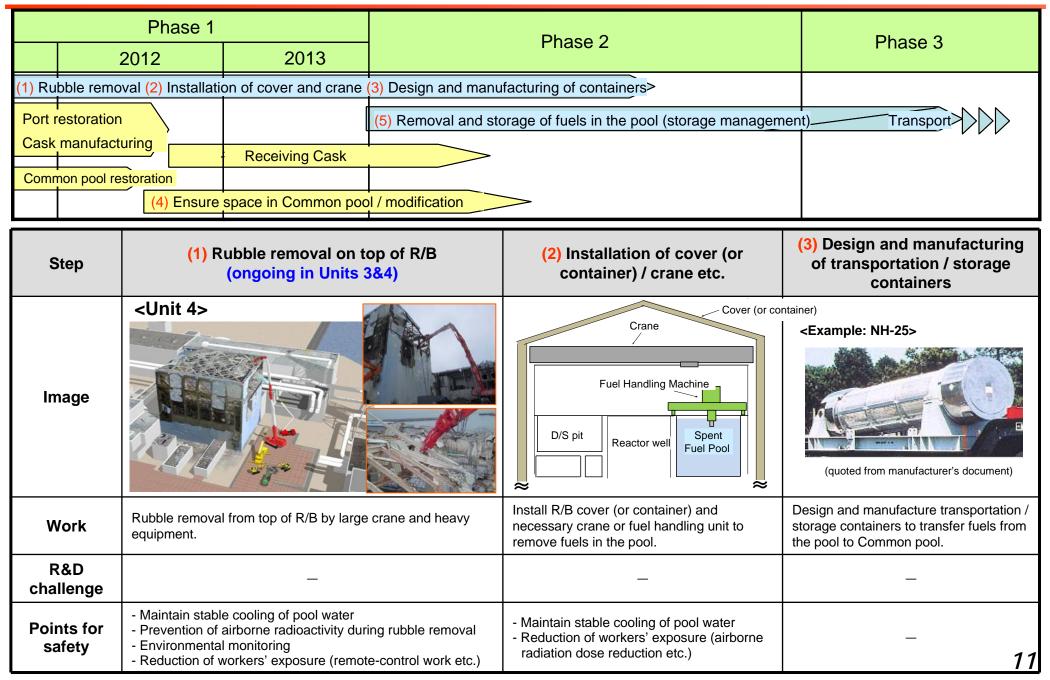


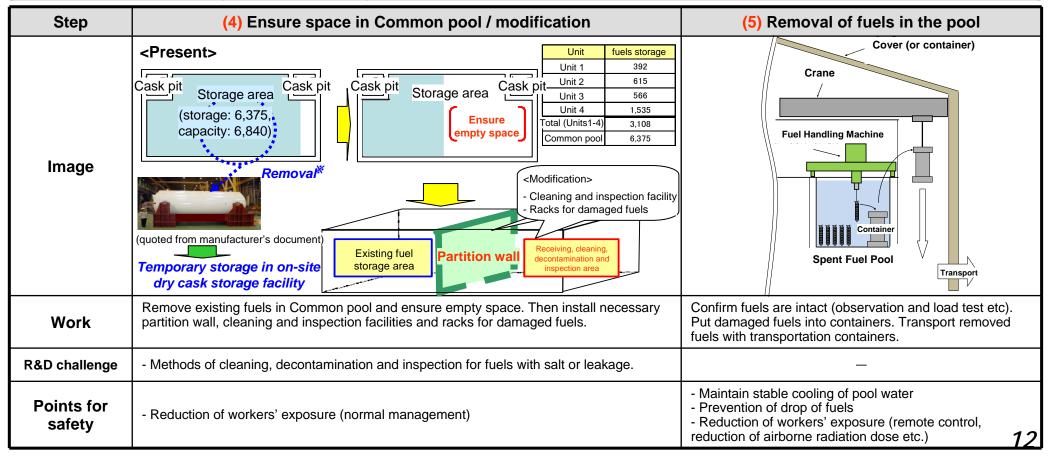
Figure 6: Fuel removal work (image)

Target Timeline: 5) Fuel Removal Step from Spent Fuel Pool (1/2)



Target Timeline: 5) Fuel Removal Step from Spent Fuel Pool (2/2)

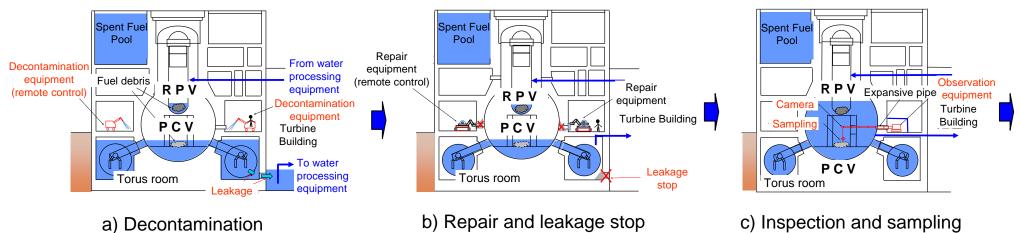
| | Phase 1 | | Phase 2 | Phase 3 | | |
|--------|---------------------------------|-----------------------|---|-------------|--|--|
| | 2012 2013 | | r nase z | Filase S | | |
| (1) Ru | bble removal (2) Installati | on of cover and crane | (3) Design and manufacturing of containers> | | | |
| | restoration | | (5) Removal and storage of fuels in the pool (storage managemen | t)Transport | | |
| | manufacturing | Receiving Cask | | | | |
| Comr | non pool restoration (4) Ensure | space in Common poc | I / modification | | | |



Target Timeline: 6) Fuel Debris Removal Plan

inside R/B

- > To start fuel debris removal in the first unit within 10 years after completion of Step 2.
- Fuel Debris Removal will be implemented in accordance with the following steps in light of the site situation, safety requirements, and R&D progress of the remote control technologies required in the operations.
- a) By the end of FY2014, start a full investigation of the leaking points while applying newly developed technologies to the site and starting decontamination of the insides of reactor buildings.
- b) By around the end of FY2015, plan to complete verification of "PCVs (lower part) repair technology" at the site. Plan to stop water leakage at the parts (lower part) identified in step "a)" by applying the new technologies. After this, the bottom part of PCVs will be flooded.
- c) By the end of FY2016, plan to complete verification of "PCVs inside investigation technology" at the site after flooding the bottom part of PCVs, and then fully investigate the insides of PCVs.



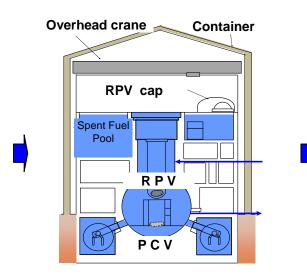
inside PCV

Fuel debris removal work (image)

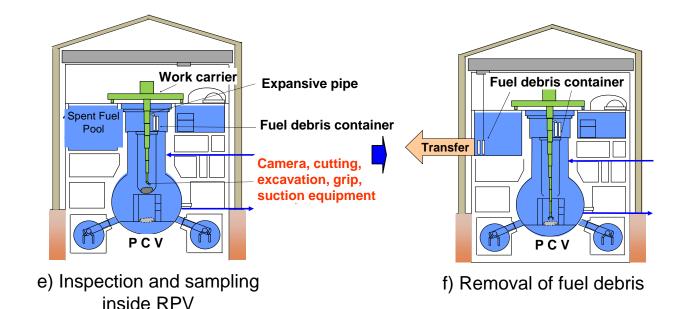
between buildings and PCV

Target Timeline: 6) Fuel Debris Removal Plan

- d) Plan to repair PCVs (upper part) and proceed with flooding. The RPV caps will be opened after installation of reactor building containers (or modified covers) to secure enclosed spaces.
- e) By around mid 2019, plan to complete verification of "RPVs inside investigation technology" at the site, and implement a full investigation of the insides of RPVs.
- f) Following the establishment of methodologies for debris removal, the development of debris containers, and the establishment of a measuring methods to weigh fuel debris based on the results of PCVs and RPVs investigations, fuel debris removal will be commenced within 10 years after Step 2 completion.

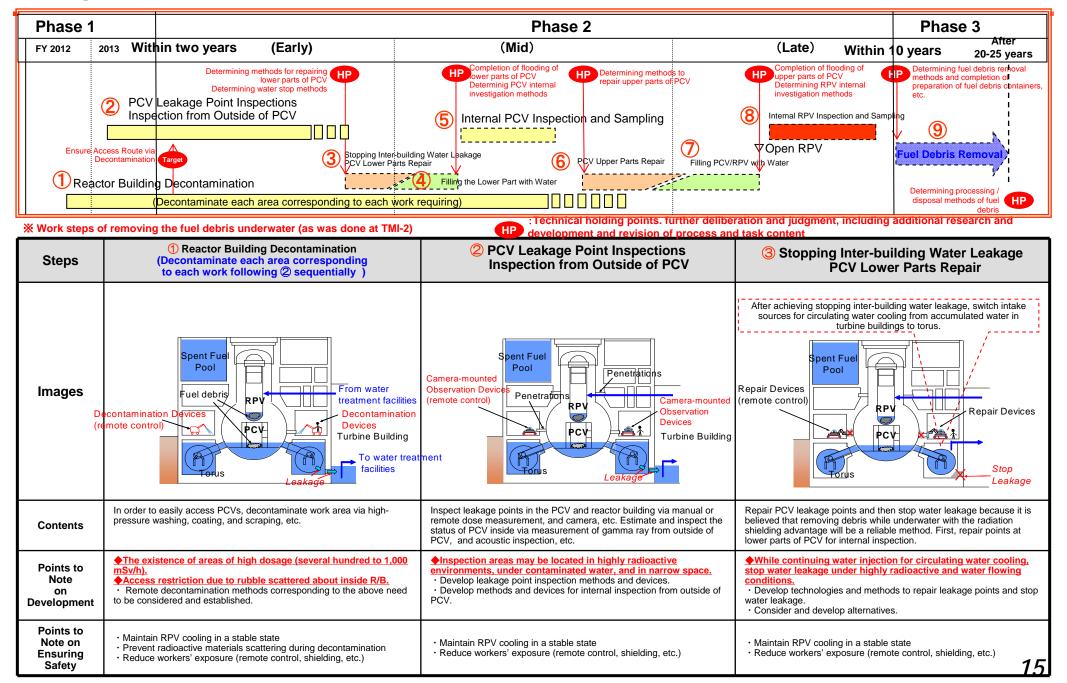


d) Flooding, opening of RPV cap

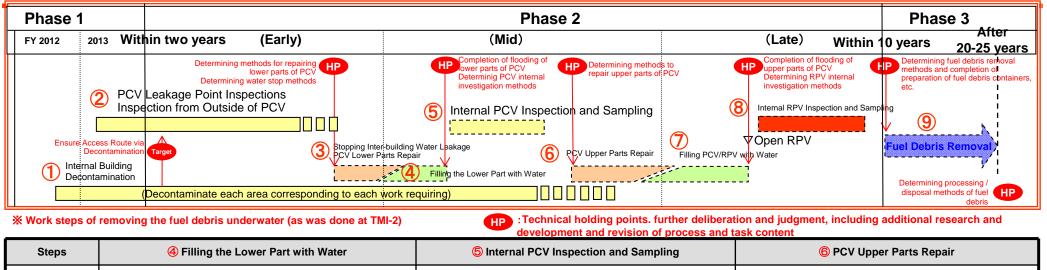


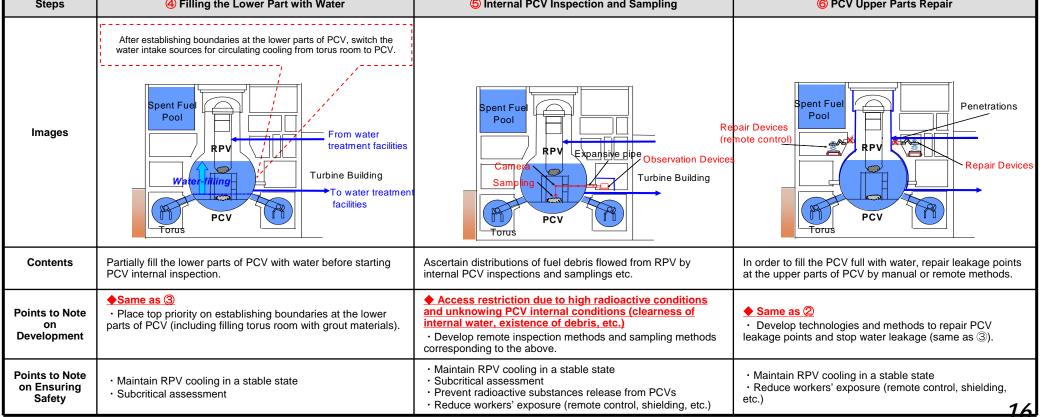
Fuel debris removal work (image)

Target Timeline: 6) Work Steps Involved in Fuel Debris Removal (1/3)

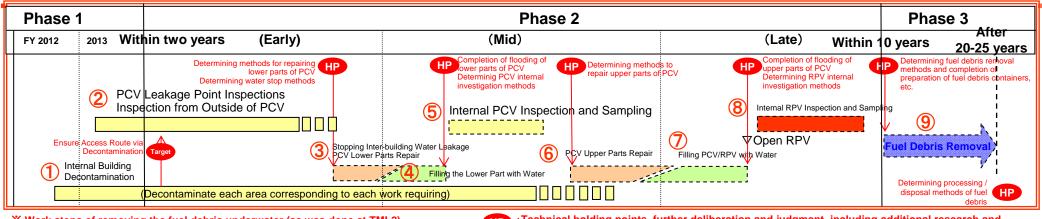


Target Timeline: 6) Work Steps Involved in Fuel Debris Removal (2/3)



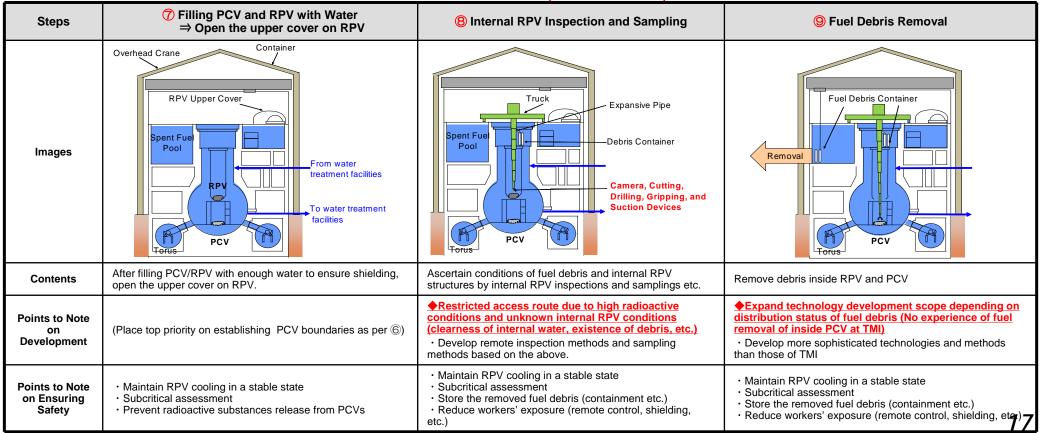


Target Timeline: 6) Work Steps Involved in Fuel Debris Removal (3/3)



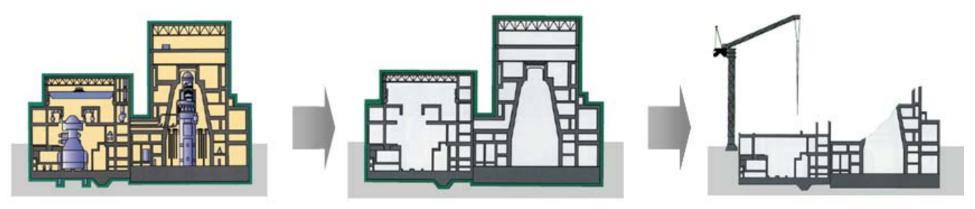
X Work steps of removing the fuel debris underwater (as was done at TMI-2)





Target Timeline: 7) Reactor Facilities Demolition Plan 8) Radioactive Waste Processing and Disposal Plan

- > Plan to complete the reactor facilities demolition in Units 1 to 4 within 30 to 40 years after the completion of Step 2.
- Plan to commence demolition in Phase 3, after confirmation of establishing a basic database of contamination necessary when considering demolition and decontamination methods, R&D progress for remote controlled demolition operations, and an outlook for the waste disposal after demolition with necessary regulatory modifications.
- Characteristic tests by heating and solidifying mockup wastes are ongoing to investigate long-term storage and disposal methodology of secondary waste from the water processing facilities (until 2013).
- Analyses of accumulated water and rubbles etc. are ongoing to estimate radionuclide concentration in wastes which is important in the process of disposal. R&D also started to develop technologies to analyze radionuclide for which appropriate methodology is not established.
- Within FY2012, plan to establish an R&D plan for the post-accident waste, whose contents differ from the ordinary waste. (nuclide composition, salt amount, etc.)
- Plan to determine waste form specifications, after confirmation of safety and applicability to the existing disposal concept as well as developing safety regulations and technical standards based on the result of R&D activities.
- Plan to commence treatment and disposal during Phase 3, after development of disposal facilities and preparation of a prospective disposal plan.



Nuclear Reactor Facilities Demolition (Image)

Target Timeline: 9) Preparation of Organization and Environment for Smooth Work

Since January 2012, on-site work has not interfered by any lack of personnel.

- Although the engaged personnel in 2012 is expected to exceed the personnel plan (approx. 11,700 persons) finally, there are approx. 24,300 persons registered for work in Fukushima Daiichi Nuclear Power Station as of May 2012 and thus lack of personnel is not expected.
- Countermeasures against heat stroke were introduced from May 2012 to secure workers' safety. Coveralls with better breathability were also introduced from late June 2012.
- Screening and decontamination facilities for vehicles started test operation in Fukushima Daiichi from April 2012 and started full-scale operation after Restricted Area in Naraha was released on August 10, 2012. New entrance management facility (for screening, protective clothing and radiation counters) will be constructed near the main entrance of Fukushima Daiichi by the end of FY2012.
- Following improper use of alarming pocket dosimeter (APD) by some workers, impact assessment on radiation control and consideration/operation of preventive measures are ongoing. Workers are continued to be instructed strictly to comply with radiation control rules.

Fundamental philosophy behind conducting the R&D

- (1) Addressing needs in the field
- This R&D aims to address necessary technical issues required to carry out the plan to remove fuels from spent fuel pools, remove fuels debris inside the reactors, and complete other steps in the process of decommissioning the plant.
- The scope of this R&D includes conducting on-site field tests since the results of this R&D will be used directly in works related to the decommissioning.
- When R&D achievements are realized, the determination as to whether or not to proceed to the next stage will be made after evaluating the feasibility and validity of technical development.
- Alternative measures will be considered in advance for challenges such as water shielding techniques that could present considerable technical hurdles.
- (2) Desired government involvement and support
- ANRE will play a leading role in R&D plan preparation and project management, and will coordinate closely with the Ministry of Education, Culture, Sports, Science and Technology as they put together an R&D framework utilizing the expertise of those in Japan and around the world.
- NISA (new regulatory agency) will implement safety regulations in accordance with the necessary legislative system for tests and demonstrations performed in the field as part of R&D.
- TEPCO assumes responsibility for field works related to the decommissioning and will be moving steadily forward with the decommissioning plan.
- (3) Open and flexible framework for actions which pool wisdom from Japan and abroad
- > Will apply technologies and knowledge of experts from Japan and other countries to the R&D.
- Will properly evaluate and assess information and advice as well as feasibility of specific cooperation from government-affiliated organizations in other countries, international organizations, and private enterprises as we build an R&D framework that is effective and efficient.

Organizational Structure for R&D

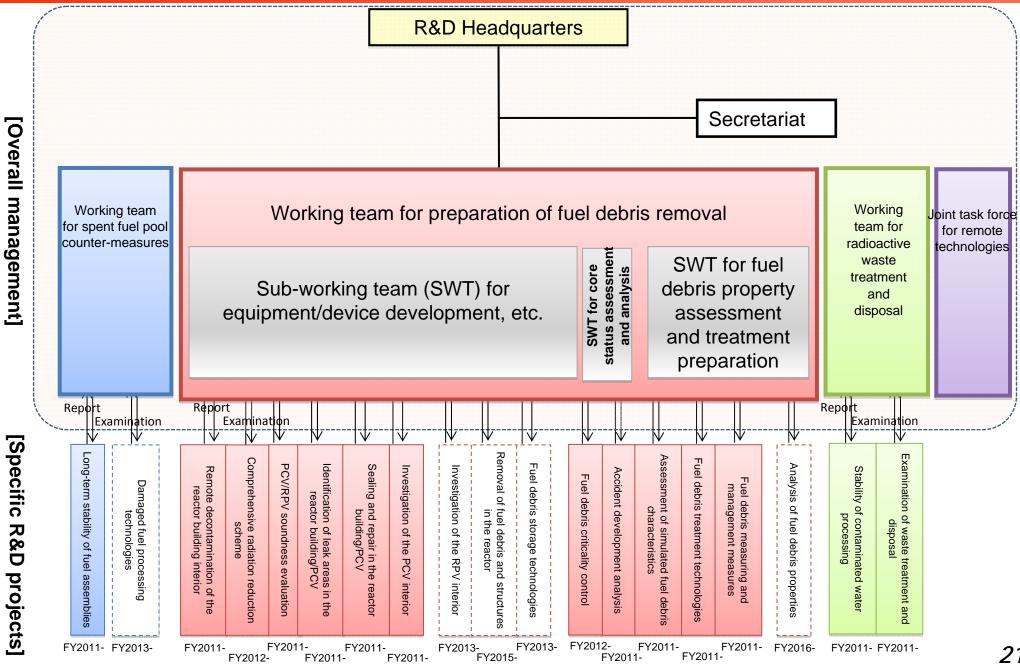


Image of Main R&D Issues related to Fuel Debris Removal

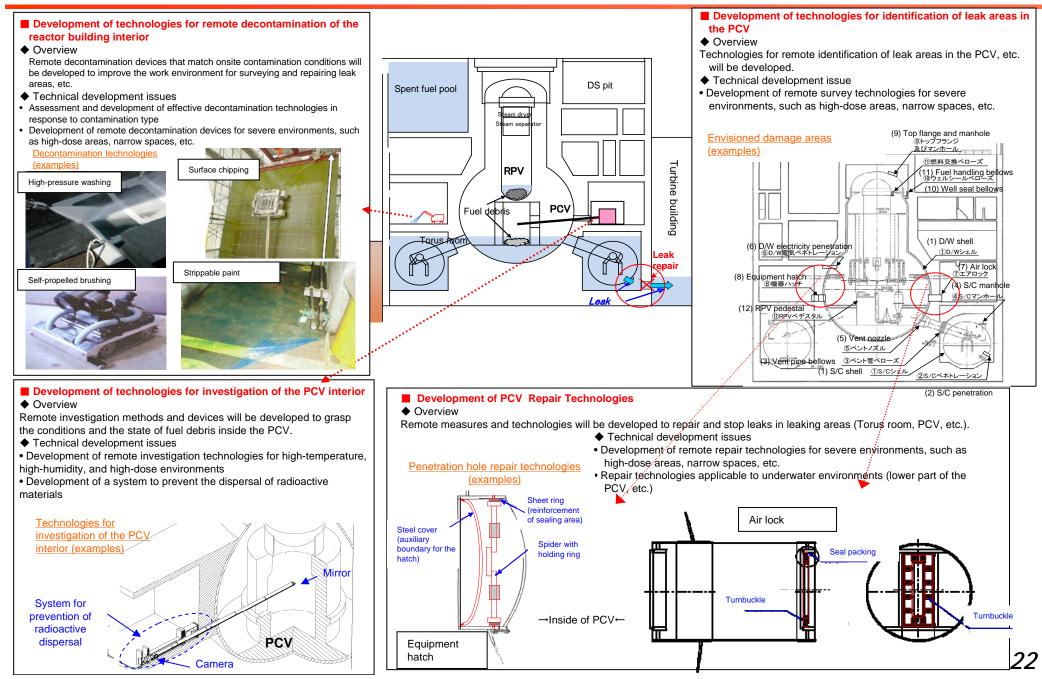
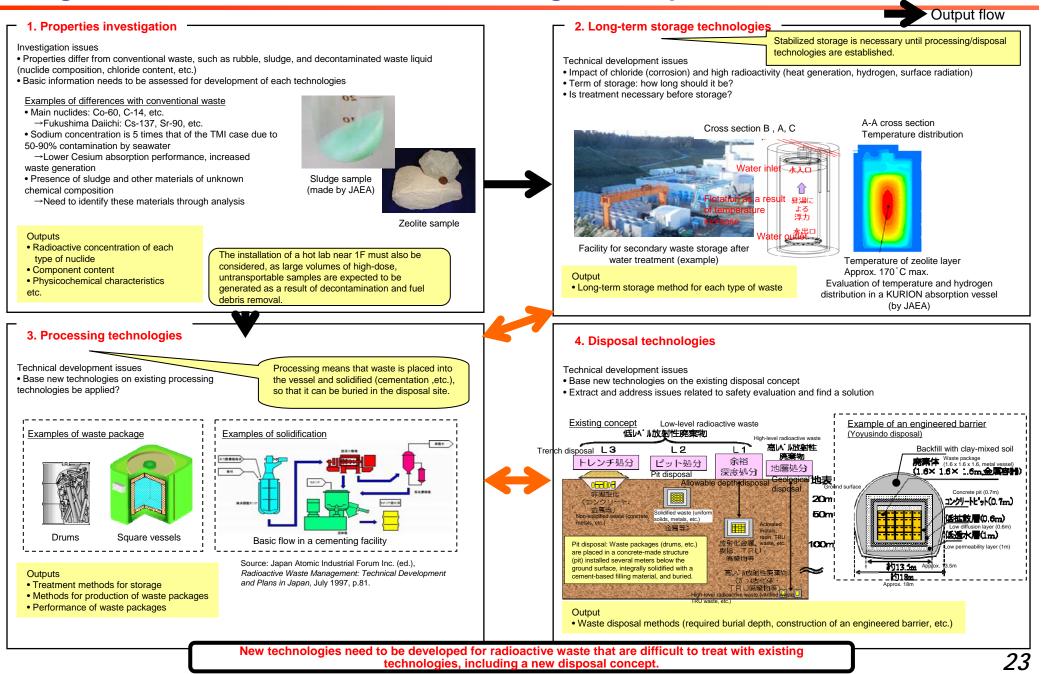


Image of R&D Issues Related to Processing and Disposal of Radioactive Waste

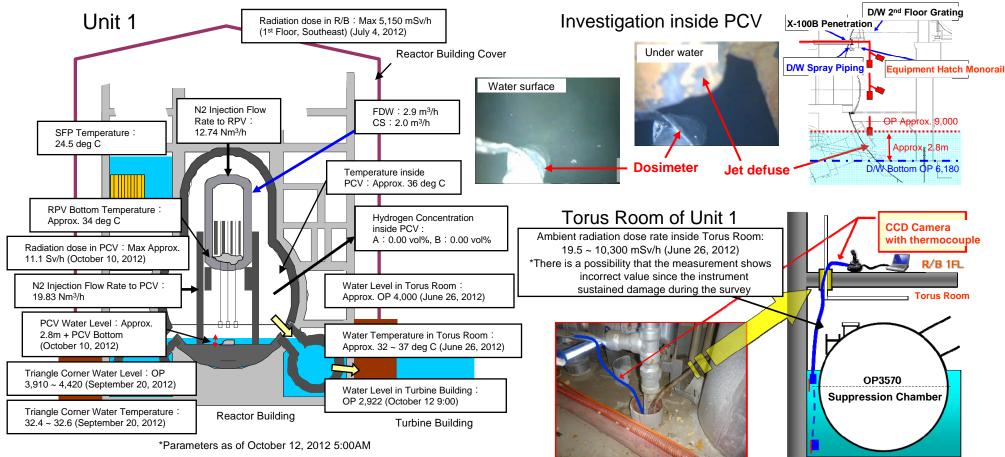


R&D started for monitoring of the reactors and removal of fuel debris

(Unit 1)

We investigated internal state of the PCV via inserting survey equipment in order to obtain photos and to directly measure data; ambient temperatures, radiation dose rate, accumulated water temperatures, water level, sampling and analysis of the water ,etc. (October 10&11, 2012).

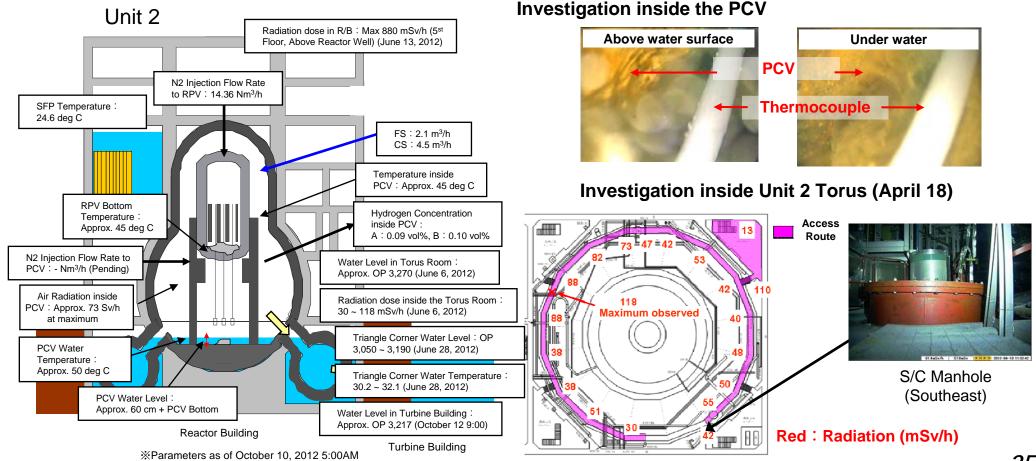
Investigation and repair of the PCV leakage is being considered. Inside of Torus Room was investigated by CCD camera inserted via penetration on the 1st floor of the Reactor Building (June 26, 2012).



R&D started for monitoring of the reactor and removal of fuel debris

(Unit 2)

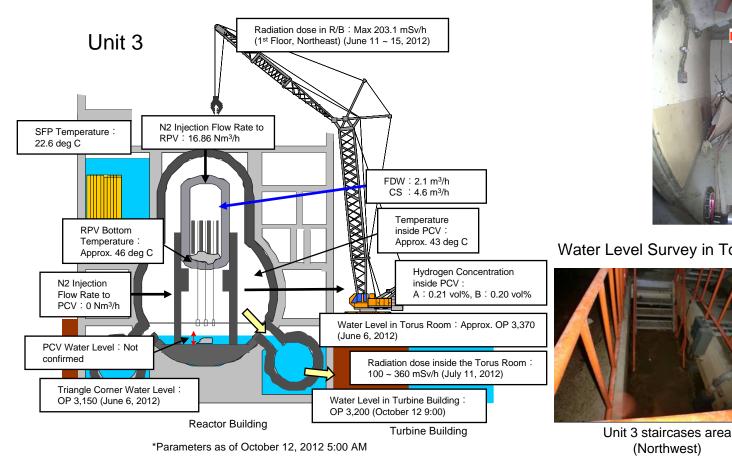
- Investigation inside was carried out with microscopes etc. via PCV penetration. (January 19 and March 26 & 27, 2012)
- Investigation inside Torus Room was carried out with robots (April 18, 2012). Water level was measured inside Torus Room and staircases area (June 6 and 28, 2012).

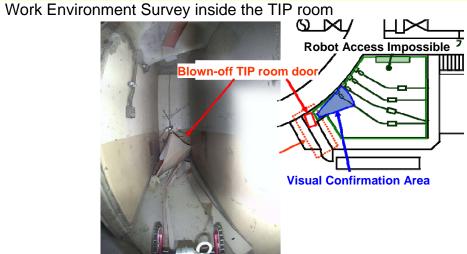


R&D started for monitoring of the reactor and removal of fuel debris

(Unit 3)

- Work Environment was investigated by robots in TIP room on 1st floor of the Reactor Building for preparation to investigate inside the PCV (May 23, 2012).
- > Water level was measured inside Torus Room and staircases area (June 6, 2012). Torus Room was investigated by a robot (July 11, 2012).





Water Level Survey in Torus and staircases area of Unit 3



| Water level | Unit 3 |
|--------------------|---------|
| Staircases area | OP 3150 |
| Torus Room | OP 3370 |

Measured water level of accumulated water